Karlsruher Institut für Technologie

Propagation of Nuclear Data Uncertainties in PWR Pin-Cell Burnup Calculations via Stochastic Sampling

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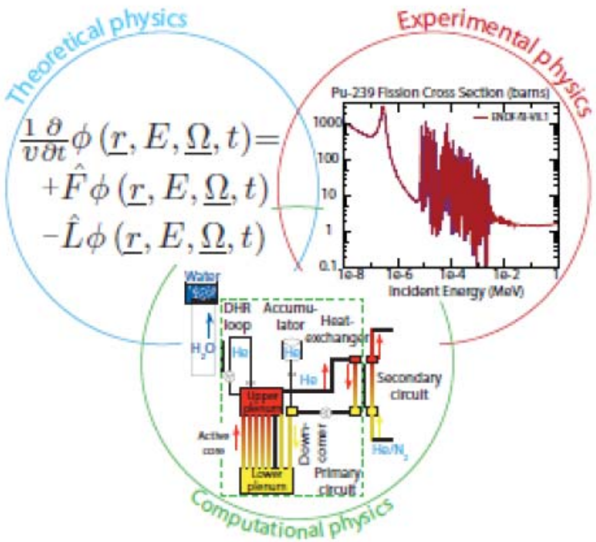
Reactor Physics and Dynamic Group (RPD)

Content

- Introduction
- S/U analysis
- Methods
  - The SAMPLER sequence
  - Problem definition and modeling
  - Calculation flowchart
  - Stochastic sampling vs. GPT methods
- Results
  - k-inf vs. irradiation time + uncertainties
  - Reaction rates
  - Nuclide concentrations
- Summary

Preamble – *"All models are wrong"*

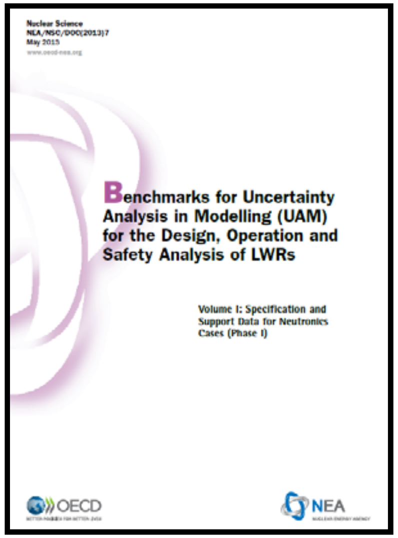
- In computer modelling, **errors and uncertainties inevitably arise** due to the **mathematical idealization** of physical processes stemming from insufficient knowledge regarding accurate model forms as well as the precise value of input parameters
- Even **the best models can only be as accurate as their input parameters**
- **Reality does not have parameters**
- Before trusting the **results** obtained by simulations one has to make sure that they are **representative of reality**



Introduction

- In recent year there has been an **increasing demand** from nuclear research, industry, safety, and regulatory bodies **for best estimate predictions of LWRs performances to be provided with their confidence bound**
- **Understanding uncertainties** of evaluated reactor parameters is important **for introducing appropriate design margins** and deciding where additional efforts should be undertaken to reduce those uncertainties

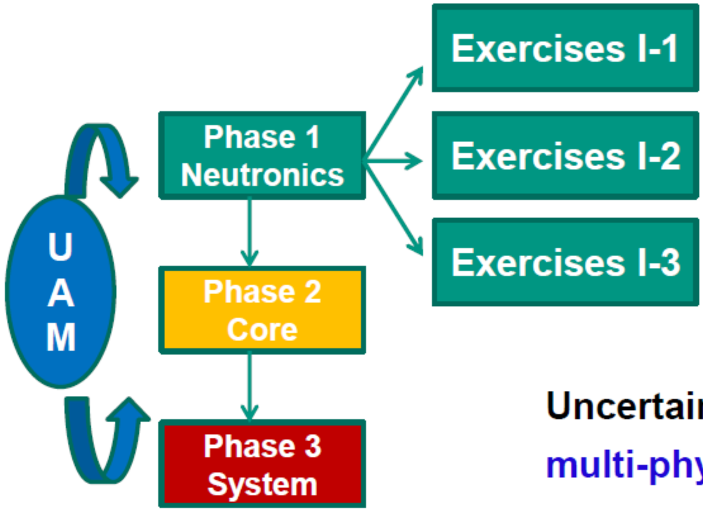
OECD/UAM Benchmark for Uncertainty Analysis in Modeling for Design, Operation and Safety Analysis of LWRs


Nuclear Science
NEA/NSC/DOC(2013)7
May 2013

Benchmarks for Uncertainty Analysis in Modelling (UAM) for the Design, Operation and Safety Analysis of LWRs

Volume I: Specification and Support Data for Neutronics Cases (Phase I)

OECD
NEA



Cell Physics: derivation of the multi-group microscopic cross-section libraries and associated uncertainties

Lattice Physics: derivation of the few-group macroscopic cross-section libraries and associated uncertainties

Core Physics: core steady state stand-alone neutronics calculations and associated uncertainties

Uncertainty propagation across multi-scale and multi-physics phenomena

S/U analysis

- **Different approaches**
  1. Direct perturbation
  2. Perturbation/Generalized perturbation theories
  3. Statistical sampling ⟸
  4. Total Monte-Carlo (TMC)

- **The statistical approach to uncertainty**
  - Uncertainty in input values described by **PDF's**
  - The **model output is a random variable** whose distribution reflects the uncertainty in the output associated with the uncertainty in the input
  - If one would know the probability distribution of the output one would be able to answer as precise as possible all questions about the likelihood of its values. The assumption of normal distribution is made
  - Statistics offers the means to "quantify the goodness" of the output values
  - Wilk's formula

$$1 - \alpha^n \geq \beta \quad (one\ sided)$$

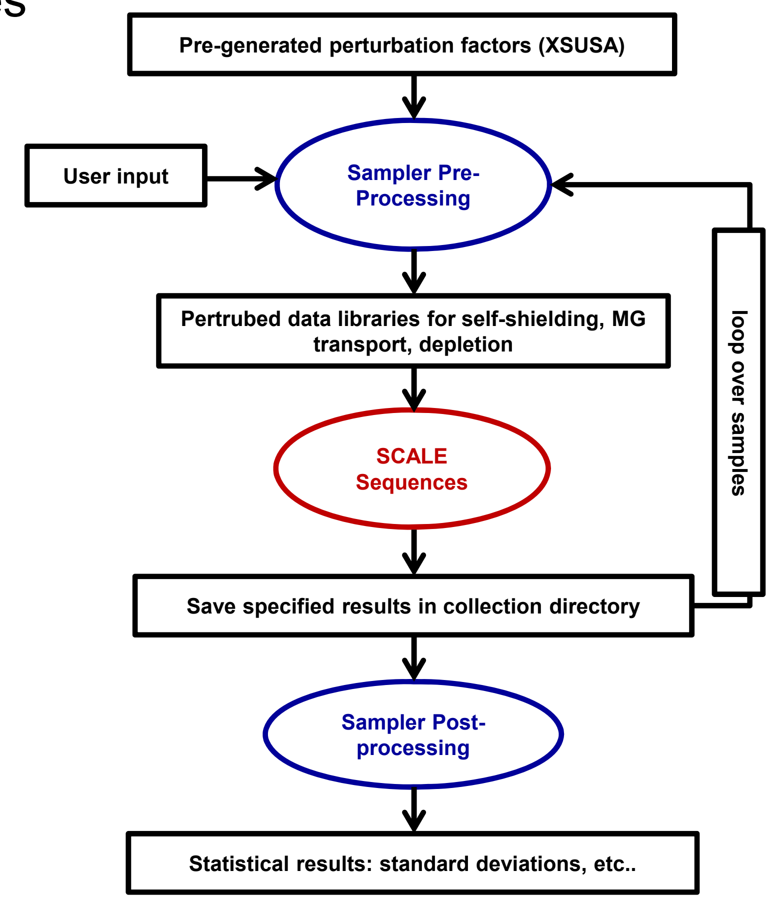
$$1 - \alpha^n - n \cdot (1 - \alpha) \cdot \alpha^{n-1} \geq \beta \quad (two\ sided)$$

	One-sided statistical limits			Two-sided statistical limits		
β / α	**0.90**	**0.95**	**0.99**	**0.90**	**0.95**	**0.99**
0.90	22	45	230	38	77	388
0.95	29	59	299	46	**93**	473
0.99	44	90	459	64	130	662

The SAMPLER sequence

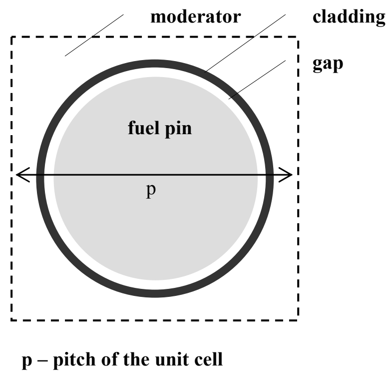
- The SAMPLER module within SCALE provides uncertainty in any computed results from any SCALE sequence due to uncertainties in:
  - Neutron cross sections
  - Fission yield and decay data
  - Geometry and composition

- SAMPLER employs sampling techniques to propagate UQ for random uncertainties
  - Given input PDF: $p(\bar{x})$
  - Given QOIs, forward model: $\bar{y} = \bar{F}(\bar{x})$
    - Compute N realizations of $\bar{x}$: $\{\bar{x}_1, \bar{x}_2, \ldots\ldots, \bar{x}_N\}$
    - Evaluate forward model of each realization $\{\bar{y}_1, \bar{y}_2, \ldots\ldots, \bar{y}_N\}$
    - Construct uncertainty quantities from sample-dependent QOI data
  - Means, SDs, correlation coefficients, histograms



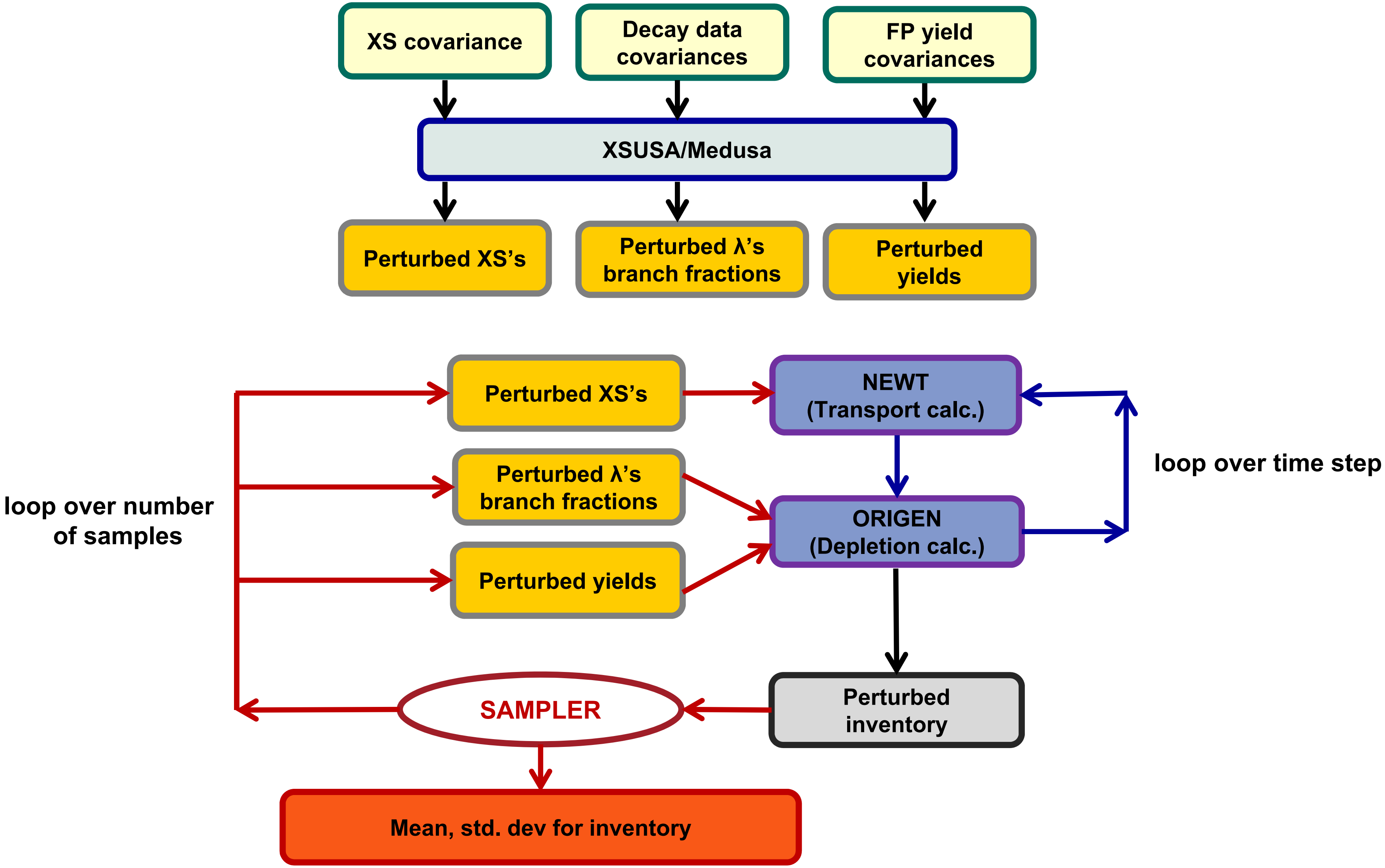
Problem definition and modeling

- **The UAM burn-up pin cell (Exercise I-1b)**
  - Power: 33.58 kW/kgU
  - Final burn-up: 61.GWd/MTU
  - Requested output
    - K-inf
    - One-group (n,f) and (n, γ) reaction rates for U and Pu isotopes
    - Actinides and FPs isotopic concentrations



- The **SCALE 6.2.2** code and **ENDF/B.VII.1** nuclear data have been used
  - **TRITON** sequence (**NEWT** + **ORIGEN-S**)
    - NEWT used to calculate weighted burn-up dependent XS's
    - BONAMI and CENTRM solvers for XS self-shielding
    - 56-group ENDF/B.VII.1 XS library
    - 56-group ENDF/B.VII.1 covariance library
  - **SAMPLER**
    - 1000 samples

Calculation flowchart



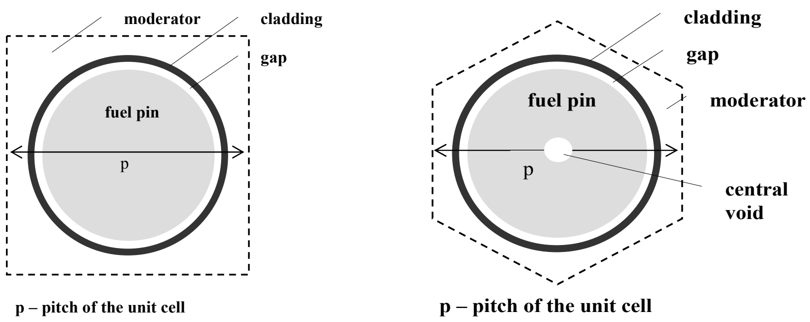
Stochastic sampling vs. GPT

- Comparison of the stochastic approach against the GPT approach showed an excellent agreement of the results(*)

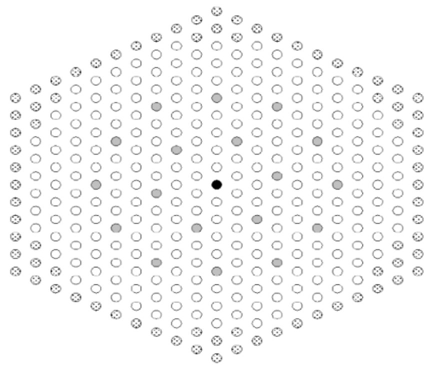
Test case	Operating condition	Calculation method			
		GPT		Sampling	
		k_{inf}	Uncert.	k_{inf}	Uncert.
BWR	HZP	1.3428	6.8E-01	1.3430	5.4E-01
	HFP	1.2249	5.9E-01	1.2252	5.7E-01
PWR	HZP	1.4253	5.4E-01	1.4254	5.0E-01
	HFP	1.4063	5.5E-01	1.4064	5.0E-01
VVER	HZP	1.3457	5.8E-01	1.3458	5.3E-01
	HFP	1.3276	5.8E-01	1.3278	5.4E-01

Test case: VVER – Kozloduy 6				
Response	TSUNAMI		SAMPLER (N=93)	
	Value	δR/R	Value	δR/R
Σ_f (gr. 1)	2,411E-03	5,071E-01	2,435E-03	5,505E-01
Σ_f (gr. 2)	5,615E-02	3,283E-01	5,701E-02	3,368E-01
Σ_a (gr. 1)	1,408E-02	1,343E+00	1,410E-02	9,076E-01
Σ_a (gr. 1)	9,485E-02	8,810E-01	9,645E-02	1,994E-01
nu-fission (gr.1)	6,159E-03	-	6,218E-03	8,418E-01
nu-fission (gr.2)	1,368E-01	-	1,390E-01	4,539E-01

Pin-cells

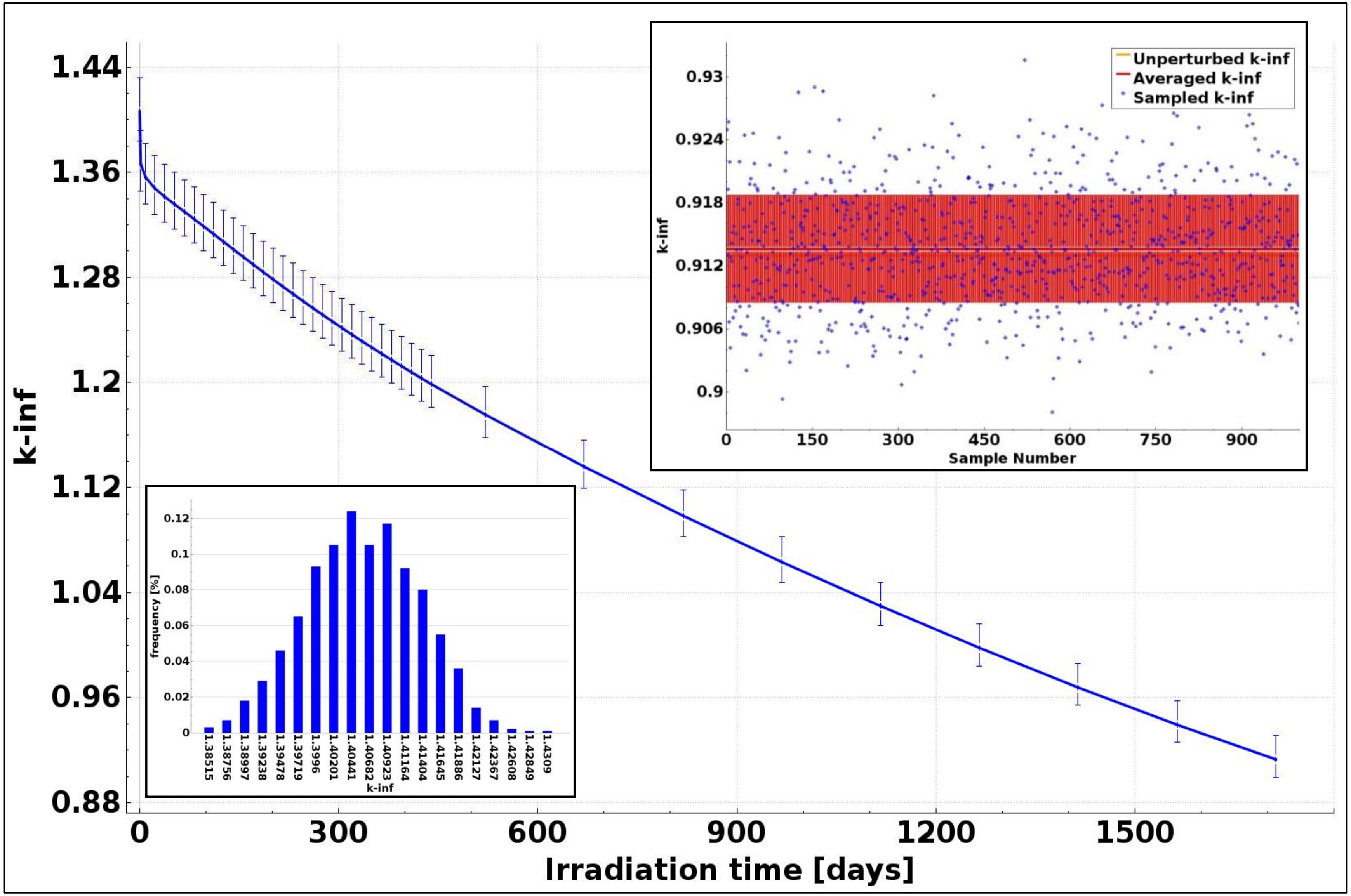


FAs



(*) L. Mercatali et al.: *"SCALE Modeling of Selected Neutronics Test Problems within the OECD UAM LWR's Benchmark", Science and Technology of Nuclear Istallations, ID 573697, Volume 2013 (2013).*

k-inf vs. irradiation time



Uncertainty on k-inf

- Three sets of 1000 samples each:
  1. XS perturbation
  2. Decay data perturbation
  3. FY perturbation

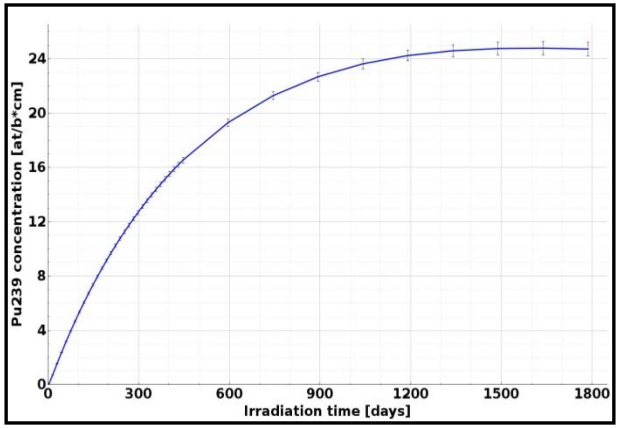
Burnup [GWd/MTU]	Perturbed case		
	XS RSD (pcm)	FY RSD (pcm)	Decay RSD (pcm)
0	545	0	0
10	503	10	4
20	495	13	3
30	494	18	3
40	505	23	3
50	527	28	2
60	560	31	2

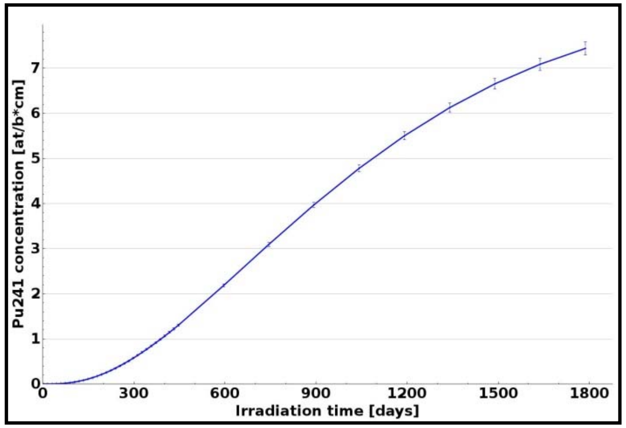
Contributions to the uncertainty on k-inf

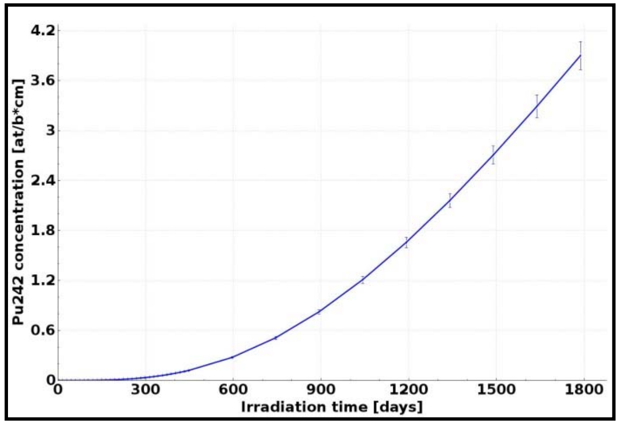
- CPU time:19,46 days on a single processor

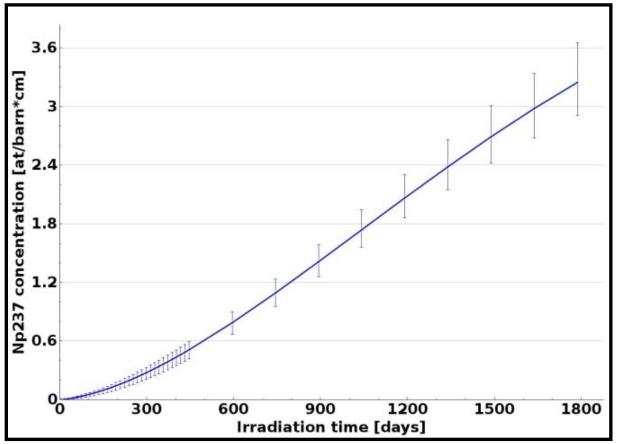
Concentrations of actinides

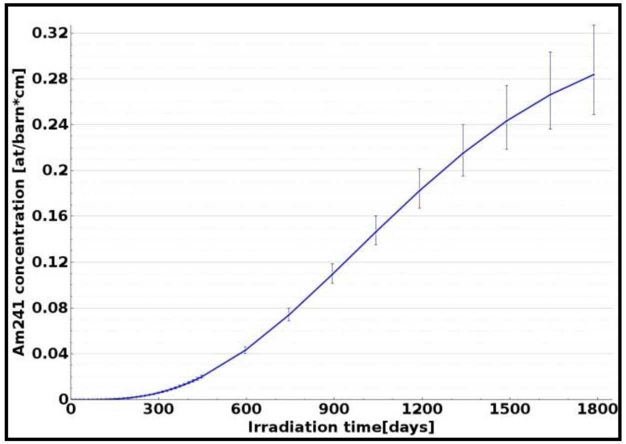
- Uncertainty mainly due to XS
- Uncertainty increases with irradiation time

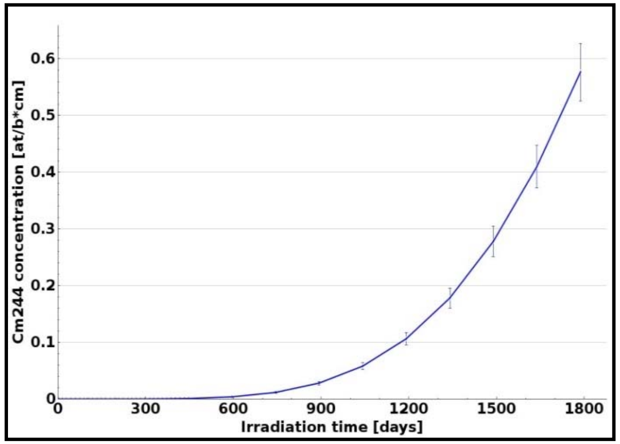




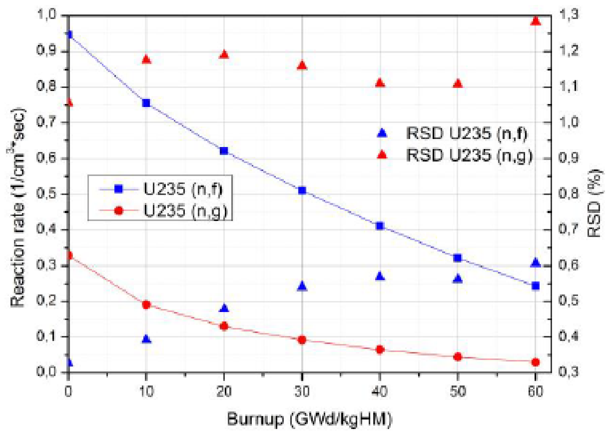


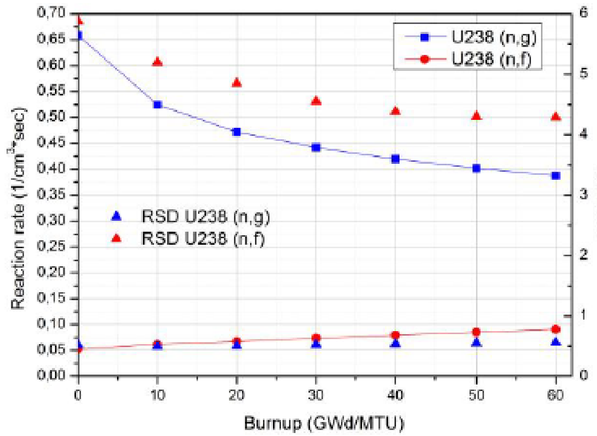


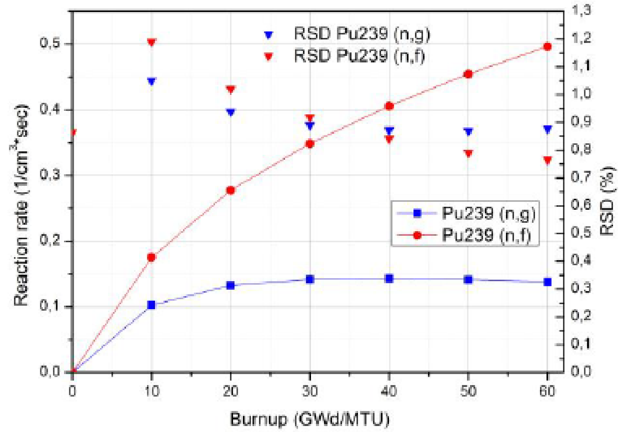


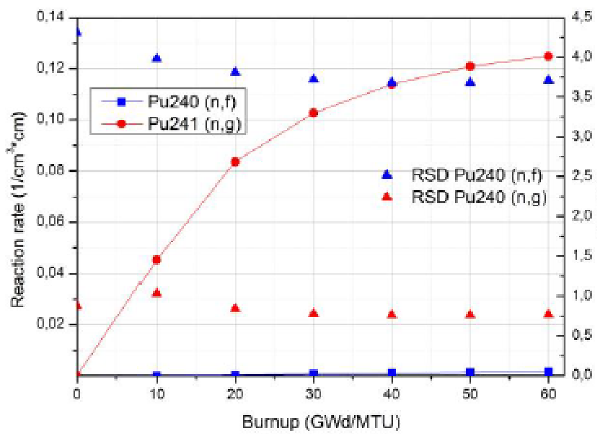


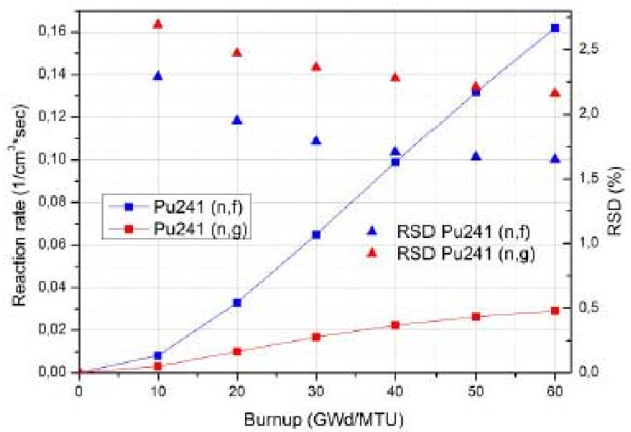
Reaction rates







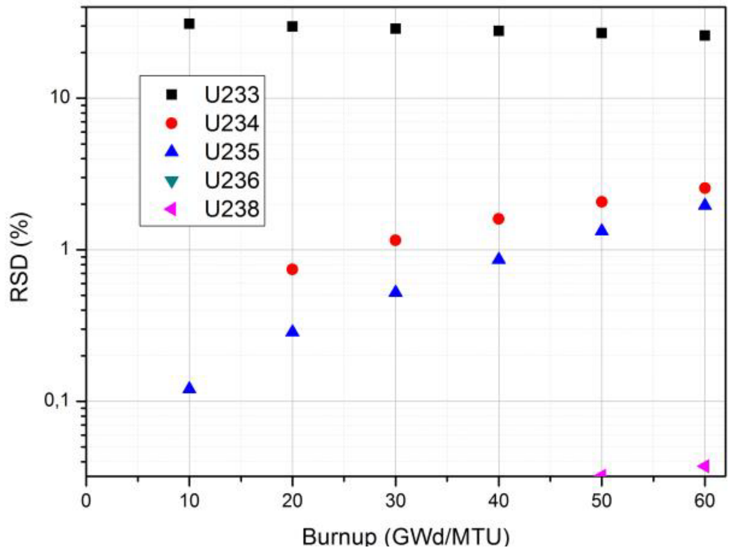


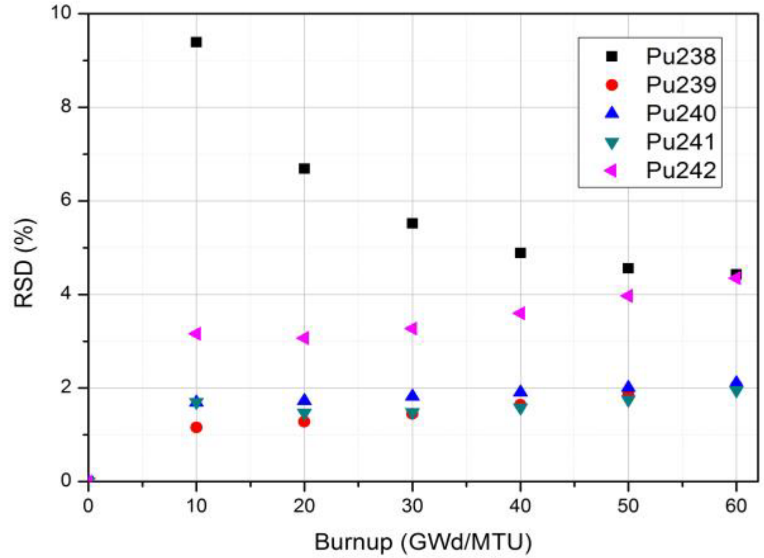


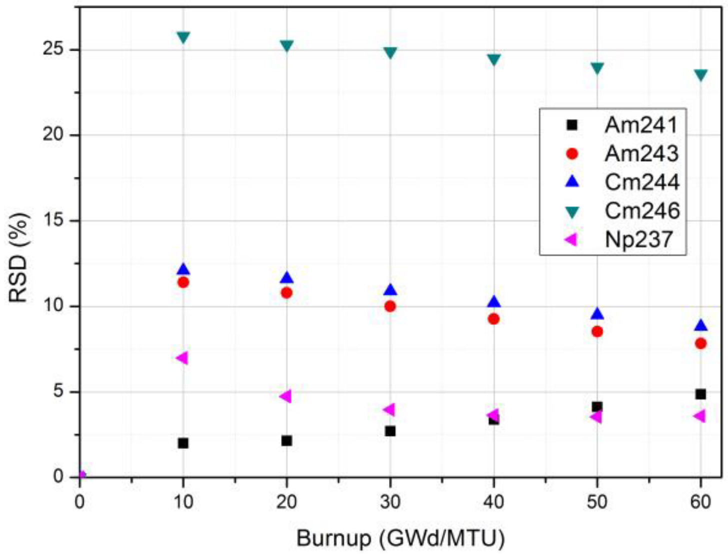
$$RSD = \frac{100 \cdot \sqrt{Var(R)}}{\overline{R}}$$

$$Var(R) = \frac{\sum_{i=1}^{N_s}\left(R_i - \overline{R}\right)^2}{N_s - 1}$$

Uncertainties on nuclide concentrations







RSD [%]	Isotopes
0 – 2	U^{235}, U^{238}, Cs^{137}, Nd^{146}, Nd^{148}, Cs^{137}
2 – 5	Pu^{238}, Pu^{239}, Pu^{240}, Pu^{241}, Pu^{242}, Am^{241}, Np^{237}, Ag^{109}, Cs^{134}, Nd^{143}, Nd^{145}, Sm^{148}, Sm^{151}, Sm^{152}, Eu^{151}, Eu^{153}, Gd^{156}, Gd^{158}
5 – 10	Am^{243}, Cm^{244}, Eu^{154}, Gd^{154}
> 20	Cm^{246}, Eu^{155}, Gd^{155}

Correlation analysis

- Pearson's coefficients (P_c)
- $\rho > 0 \rightarrow$ correlation, $\rho < 0 \rightarrow$ anti-correlation
- $|\rho| = 1 \rightarrow$ perfect linear relationship

$$P_c = \frac{Cov(x,y)}{SD(x) \cdot SD(y)} \in [-1,1]$$

0 GWd/MTU									
k_{inf}	1.000								
$\Sigma_{a,1}$	-0.488	1.000							
$\Sigma_{a,2}$	-0.218	0.170	1.000						
$\Sigma_{f,1}$	0.405	-0.073	0.083	1.000					
$\Sigma_{f,2}$	0.345	0.038	0.415	0.319	1.000				
$\nu\Sigma_{f,1}$	0.630	-0.279	0.059	0.808	0.187	1.000			
$\nu\Sigma_{f,2}$	0.696	0.061	0.291	0.213	0.617	0.403	1.000		
D1	0.291	-0.858	-0.038	-0.001	-0.053	0.271	-0.053	1.000	
D2	-0.021	-0.146	-0.007	-0.129	-0.053	-0.078	-0.061	0.245	1.000

60 GWd/MTU									
k_{inf}	1.000								
$\Sigma_{a,1}$	-0.600	1.000							
$\Sigma_{a,2}$	0.051	0.401	1.000						
$\Sigma_{f,1}$	0.539	-0.262	0.678	1.000					
$\Sigma_{f,2}$	0.280	0.348	0.942	0.682	1.000				
$\nu\Sigma_{f,1}$	0.558	-0.265	0.638	0.972	0.636	1.000			
$\nu\Sigma_{f,2}$	0.275	0.368	0.944	0.667	0.996	0.625	1.000		
D1	0.610	-0.779	0.127	0.678	0.163	0.658	0.143	1.000	
D2	0.087	-0.291	-0.267	-0.151	-0.239	-0.143	-0.244	0.227	1.000

Summary

- A cell physics exercise has been performed, aiming to assess the uncertainties associated with the basic nuclear data in burn-up calculations for a typical PWR fuel pin-cell through a stochastic sampling approach
- Results obtained with the stochastic sampling method are in very good agreement with the ones obtained via GPT
- Uncertainties have been quantified as a function of the depletion time
- FY and decay constants have a negligible impact on the total uncertainty, the main contributor being the XS uncertainty
- The study represents the first step towards the uncertainty quantification for more complex burn-up problems (FAs, full core)